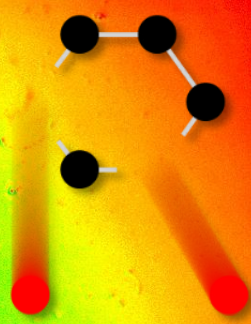


The 6th RaDIATE Meeting, TRIUMF
Radiation Damage Studies II
(11:30-11:55, Dec. 10th, 2019)



Status of Ion Beam Irradiation at HIT facility

E. Wakai¹

S. Kano²

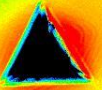
T. Ishida^{1*}

S. Makimura¹

H. Abe²

1. J-PARC

2. Univ. of Tokyo



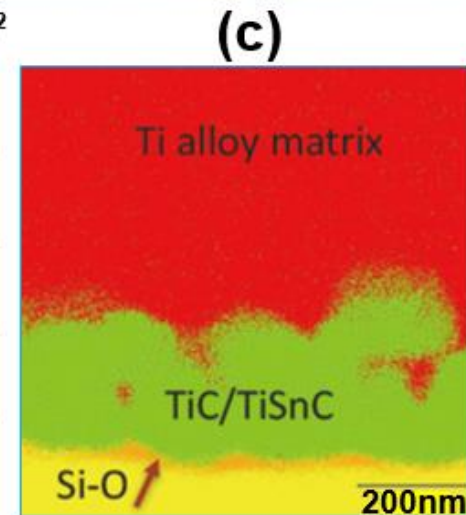
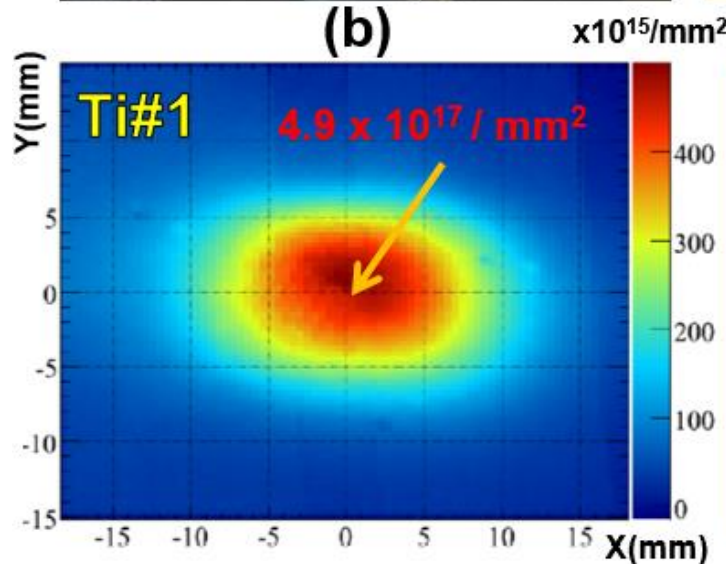
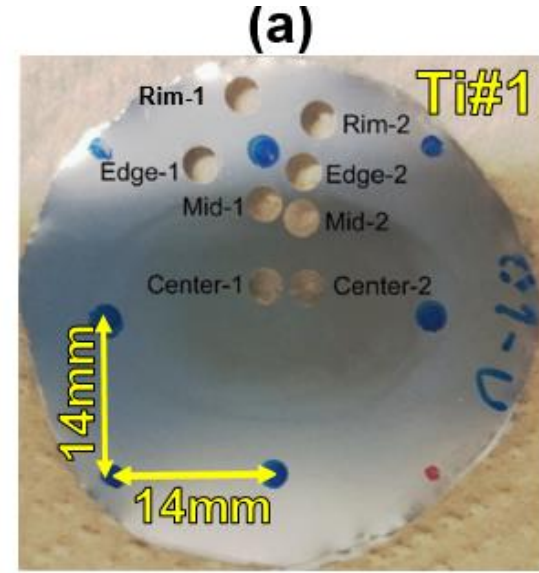
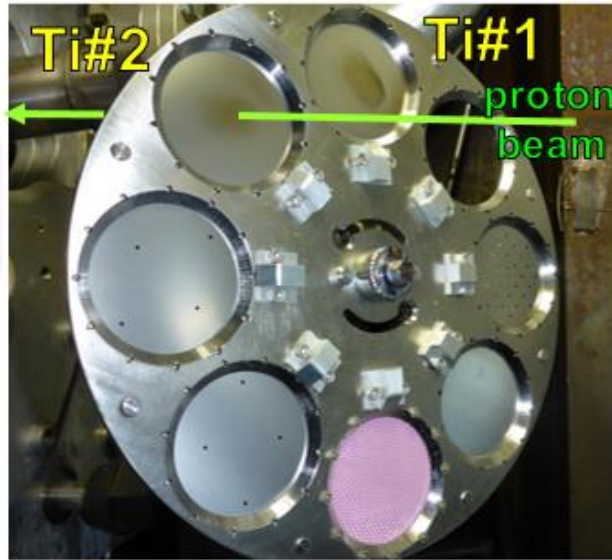
Background -1

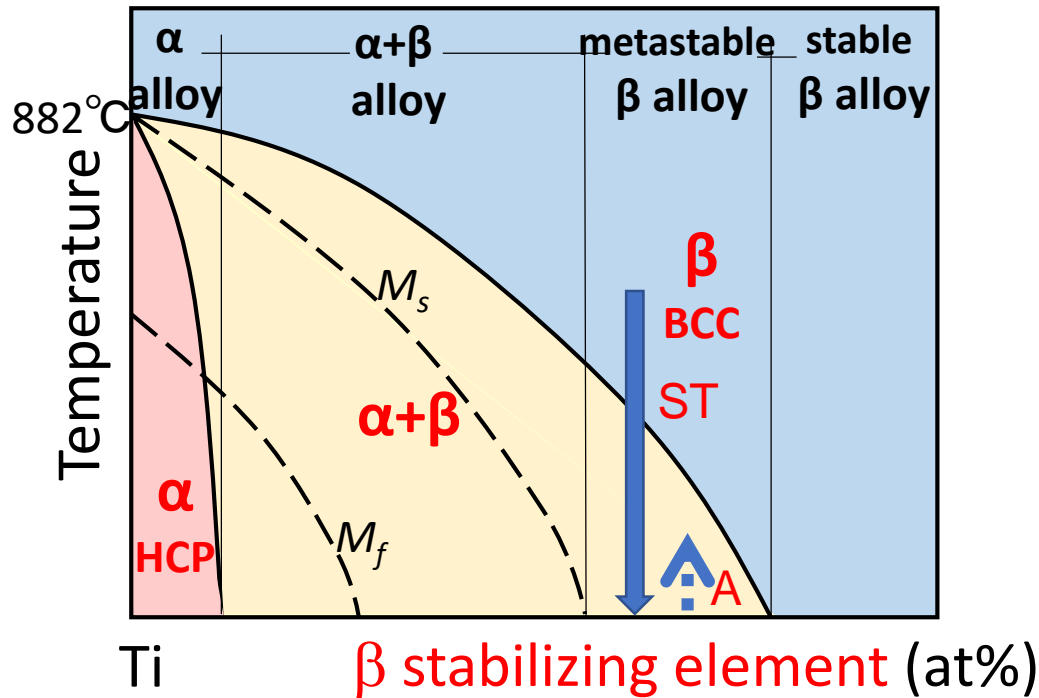
T. Ishida, E.Wakai, D.Senor, A.Casella, D.Edwards et al.,
[Nuclear Materials and Energy 15 \(2018\) 169-174.](#)

J-PARC OTR: 50 μm -thick metastable- β (BCC) Ti-15V-3Al-3Cr-3Sn (ST) Irradiation Damage Analysis at PNNL

Ti-1: 1.4×10^{20} pot
0.12 peak DPA-NRT

(Ti-2: 5.2×10^{20} pot)
(0.29 peak DPA)



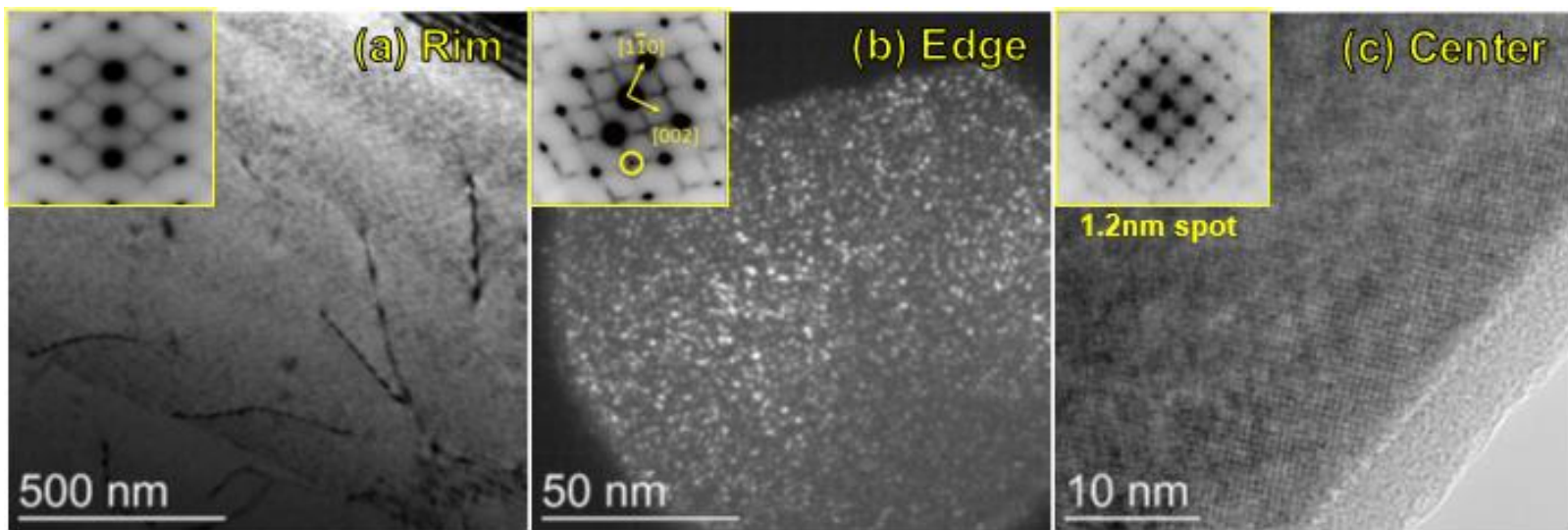


$$\text{Mo equivalent (mass\%)} = \text{Mo} + 0.67 \times \text{V} + 0.44 \times \text{W} + 0.28 \times \text{Nb} + 0.22 \times \text{Ta} + 2.9 \times \text{Fe} + 1.6 \times \text{Cr} - 1.0 \times \text{Al}$$

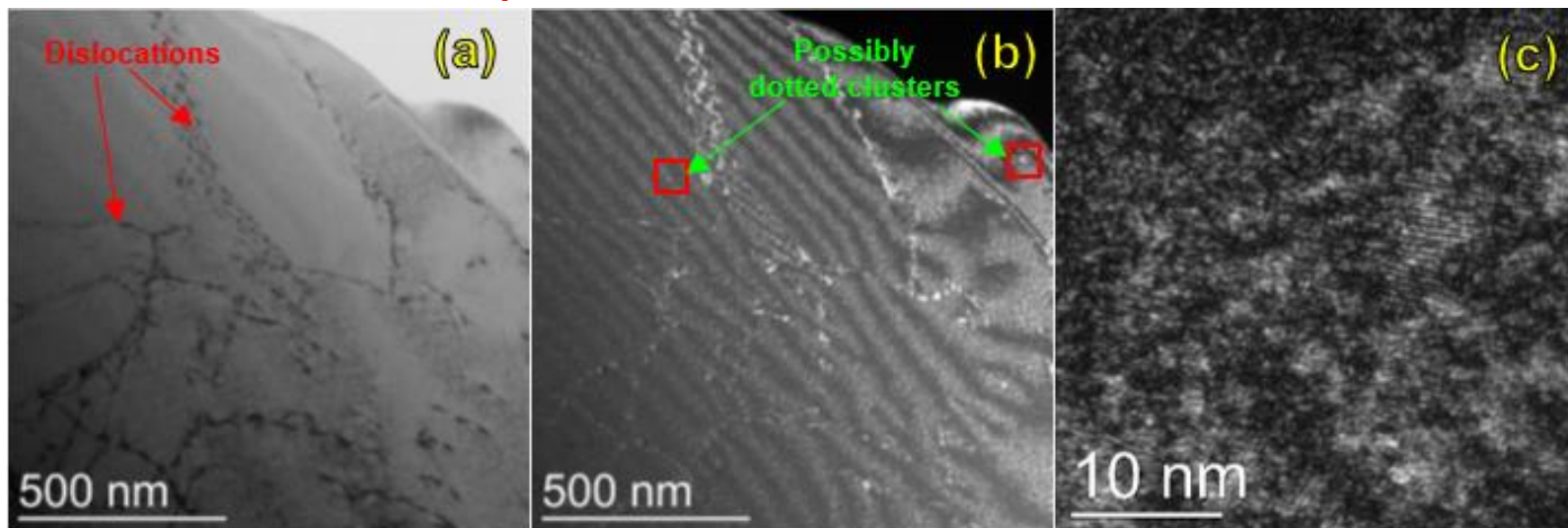
- The $\alpha+\beta$ phase and metastable β phase alloys are heat-treatable
- Solution Treatment and Aging (STA)**: maintain the alloy at a temp. higher than the β transus to dissolve the alloying elements
- Quench it to keep in a metastable supersaturated solid solution state.
- Aging at an elevated temp. for several hrs. generates **a fine (less than 10 nm) scale precipitation in β -phase grains**, i.e., α phase for aging above $\sim 500^\circ\text{C}$, or ω -phase for aging between $400\sim 500^\circ\text{C}$.
- Whilst the fine scale precipitation makes the alloy stronger **“precipitation hardening”**, there is evidence to suggest that the ω precipitation can lead to embrittlement **“ ω -embrittlement”**.

Grade	Heat Tr	0.2% YS	TS	TE
Ti-15V-3Cr-3Al-3Sn (metastable β)	ST	790 MPa	860 MPa	11%
	STA	1,280 MPa	1,400 MPa	7%
Ti-6Al-4V ($\alpha+\beta$)	Anneal	900 MPa	1,000 MPa	13%
	STA	1,050 MPa	1,160 MPa	5%

→ T.Ishida et al, <https://arxiv.org/abs/1911.10198>

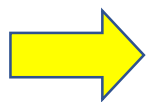


Before irradiation, microstructures of 15-3 Ti exhibited a high density of nano-clusters of athermal ω and martensitic α' phases. These were stable after irradiation.



Very low density of defect clusters were formed by irradiation about 0.06 dpa.

No significant changes on micro-Vickers hardness



15-3 Ti alloy may have a high resistance for radiation damage, due to nano-scale precipitates working as *point-defect sink-sites*

Background -2

■ Correlation of Radiation Hardening Measurement Method

cm scale

mm scale

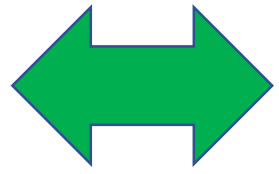
Micro -

Nano scale

$\sigma_y(\text{MPa}) \doteq 3 \text{ Hv}$

$\text{Hv} \doteq 60 \times \text{Hm}(\text{GPa})$

Tensile Data

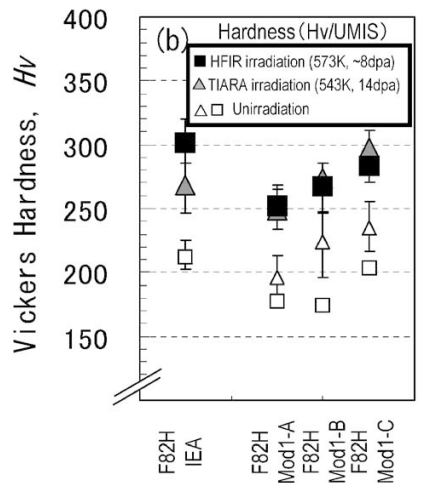
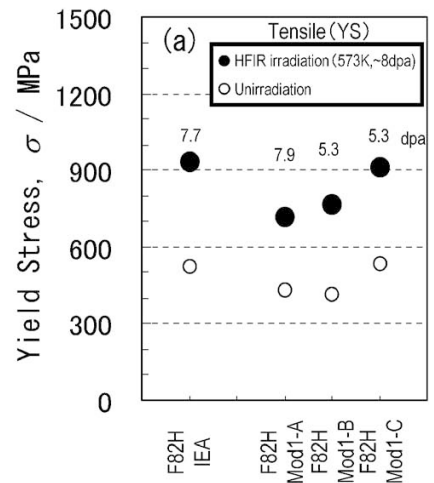


Vickers Hardness Data

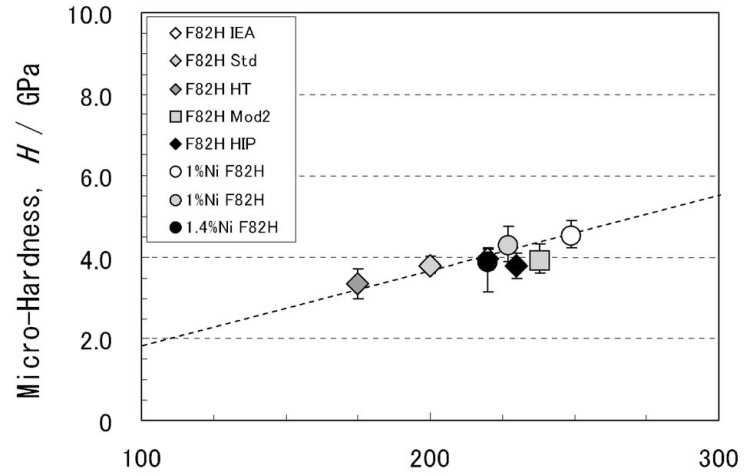


Micro-Hardness Data

Correlation-1



Correlation-2



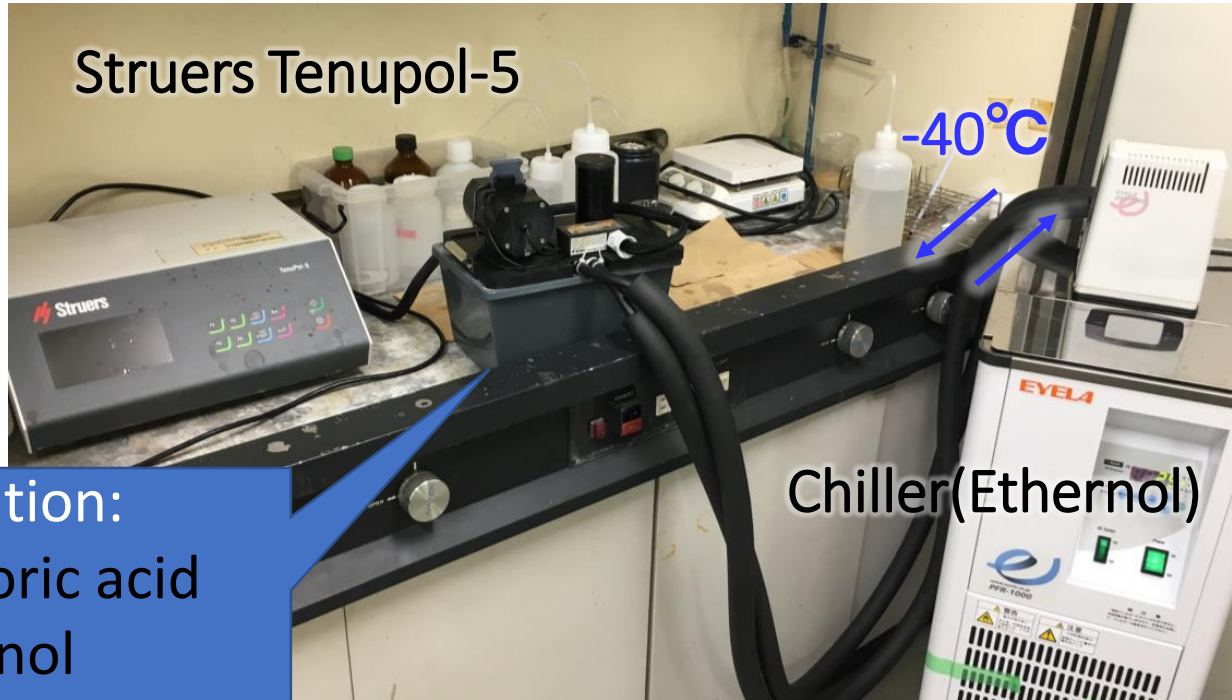
(M. Ando, et al., *J. Japan Inst. Metals*, Vol. 72, No. 10 (2008), pp. 785-788.) Vickers Hardness, Hv

→ Low energy ion beam irradiation is useful in the evaluation of hardening behavior at high DPA region

Objective

- 1. Complement studies with high energy proton irradiation experiments**
- 2. Estimate hardening behaviors in high DPA region and obtain DPA dependence for different alloys**
- 3. Select material grades and heat treatments with higher radiation damage resistance**

Twin-jet Electro Polishing



Solution:

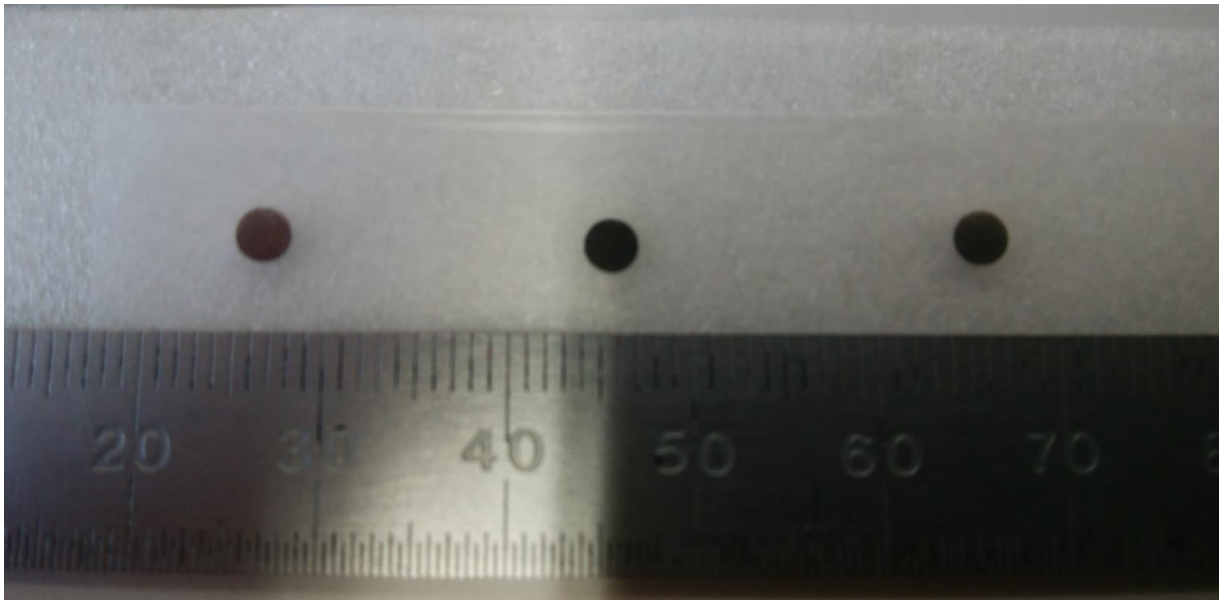
6% Perchloric acid

35% Butanol

59% methanol

Chiller(Ethernol)

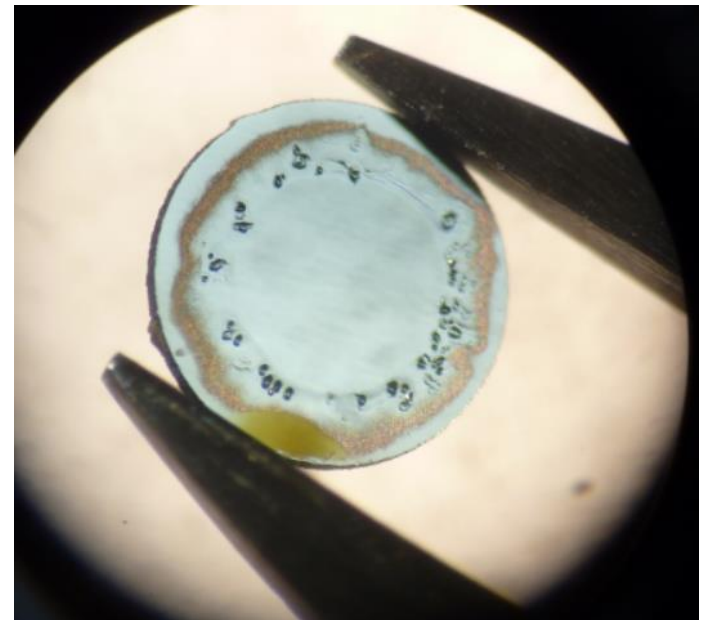
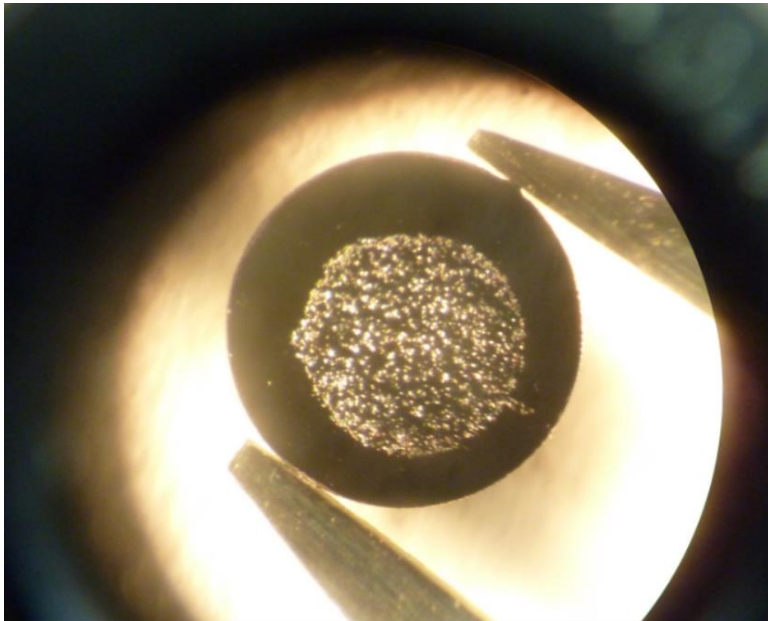
Specimen	Electro-polishing condition
Ti-15V-3Cr-3Al-3Sn (Metastable β) Solution Treatment, not Aged (ST)	-40V, flow rate 22, 60-68 mA
Ti-15V-3Cr-3Al-3Sn (Metastable β) Solution Treatment and Aged (STA)	-40V, flow20, 10sec, 68 mA
Ti-64 ($\alpha+\beta$) Annealed (A)	-60V, flow22, 10sec, 106 mA
Ti-64 ($\alpha+\beta$) STA (WQ -> Aging)	-50V, flow22, 10sec, 88 mA



$\Phi 3 \times 0.2\text{mm t}$

Pitting Damage (Not Used)

Good Condition

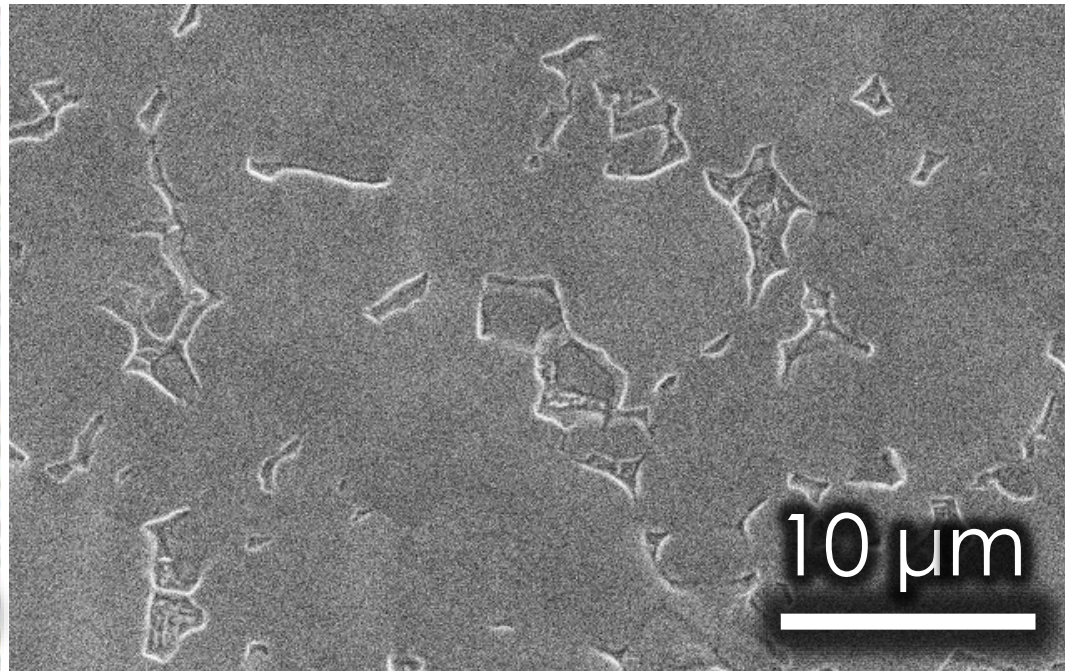
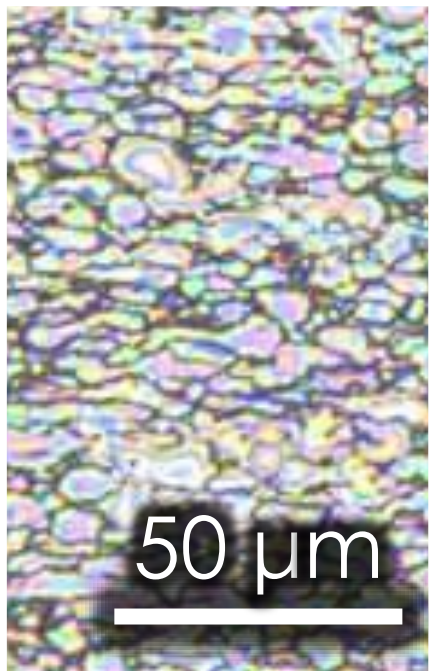


6-4

Heterogeneous
corrosion spots

Precedential
corrosion on
beta phases

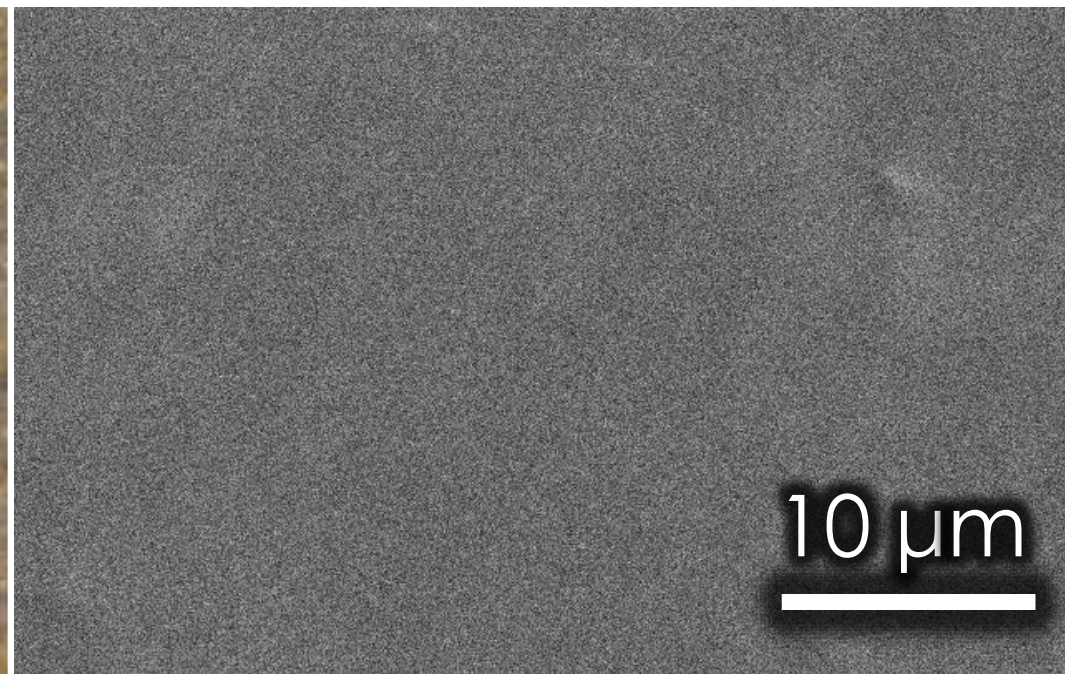
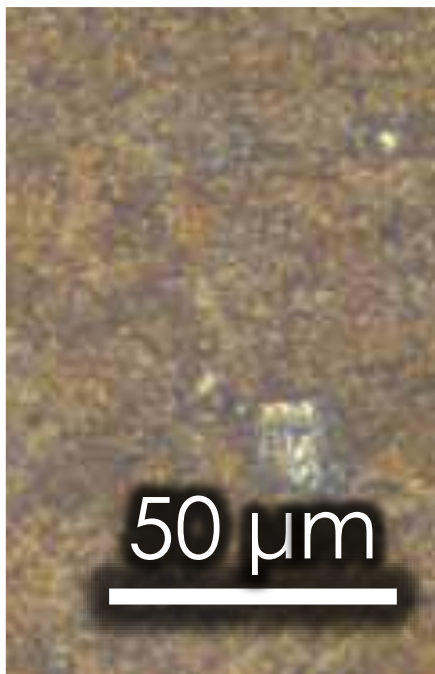
Depth: ~200nm



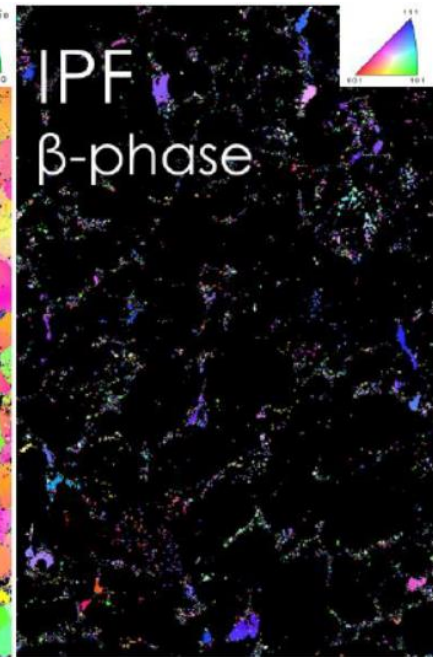
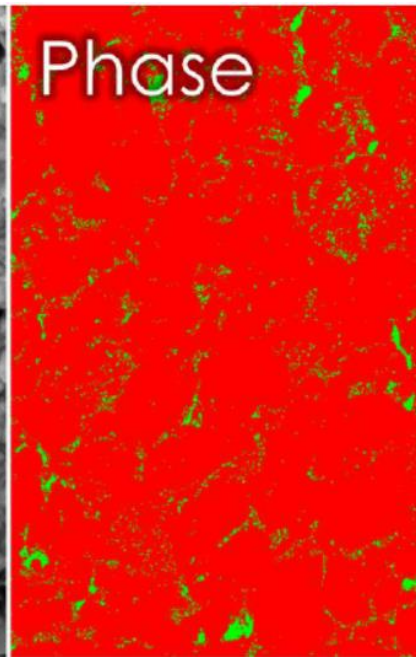
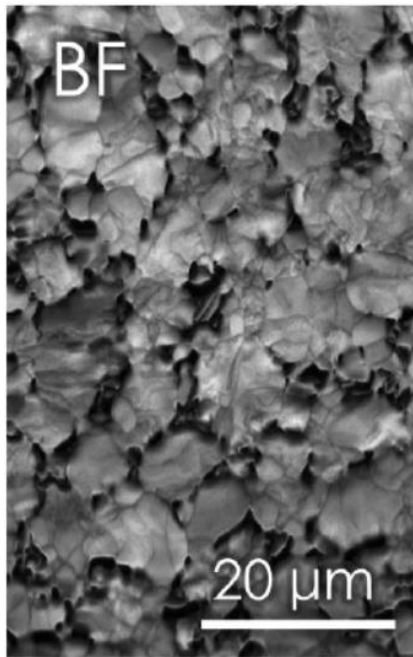
15-3

Smooth surface

No contrast

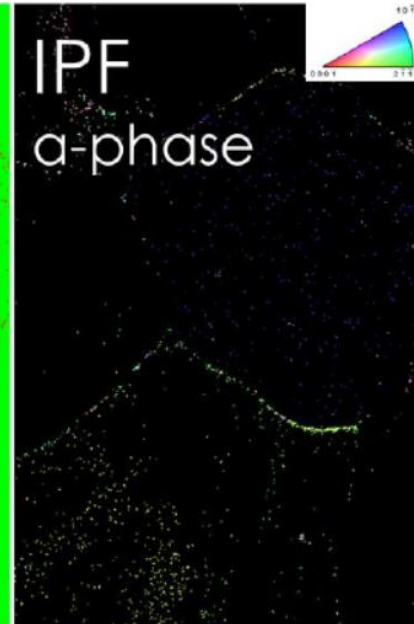
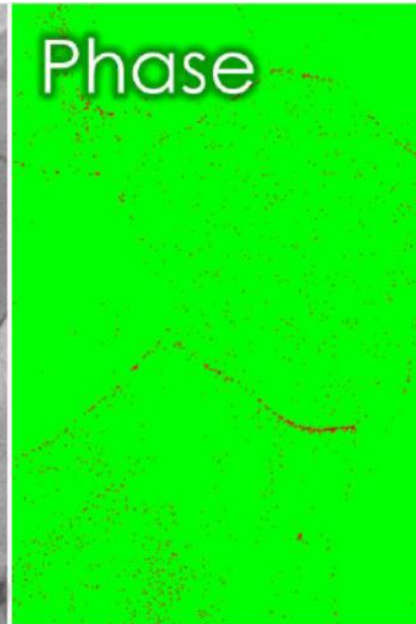
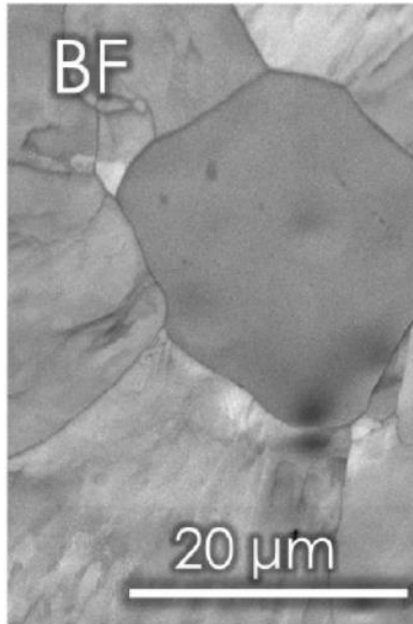


6-4



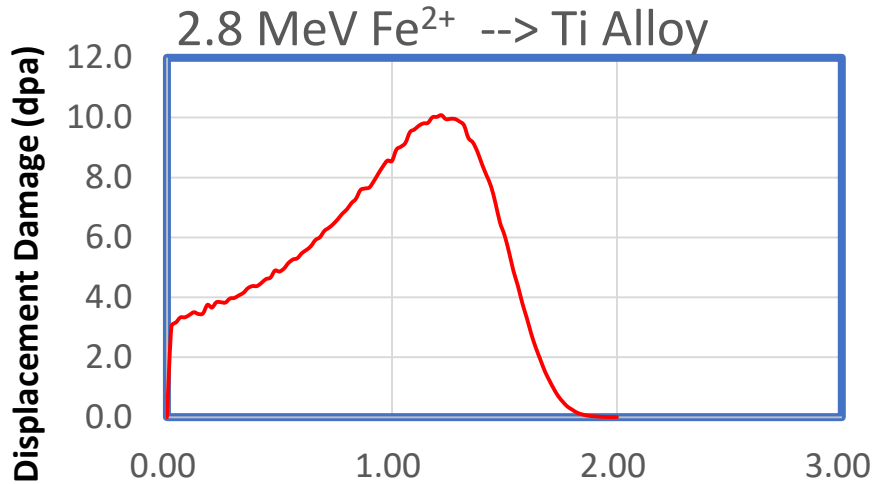
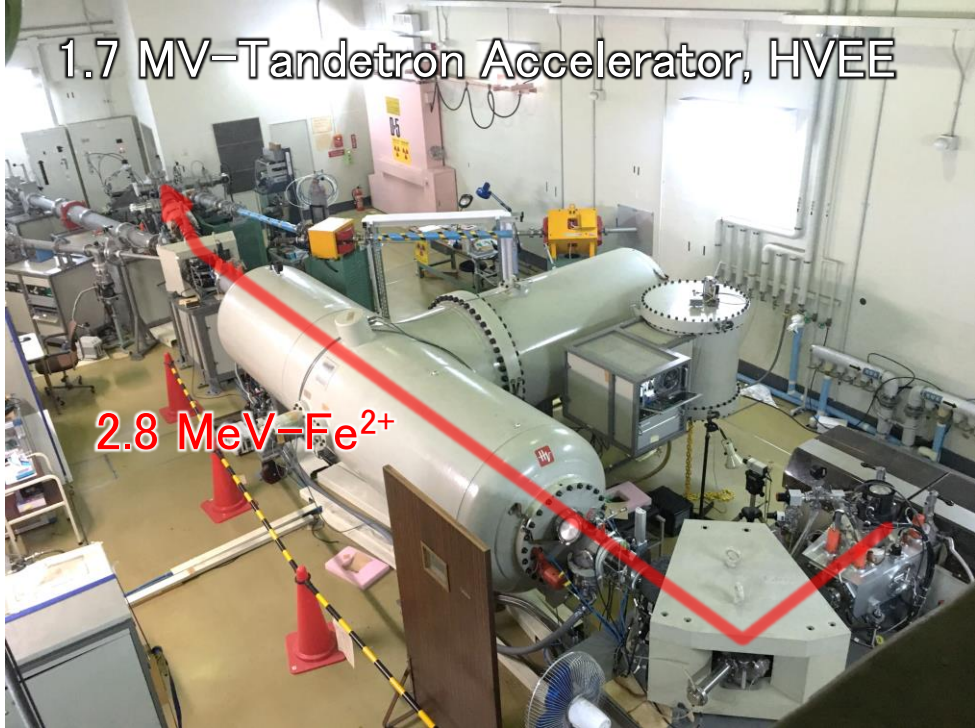
α : 95.2%
 β : 4.8%

15-3

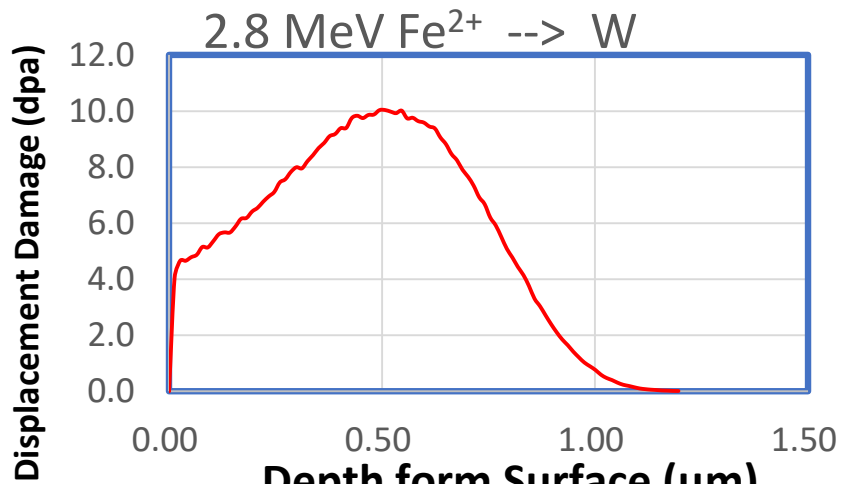


α : 1.1%
 β : 98.9%

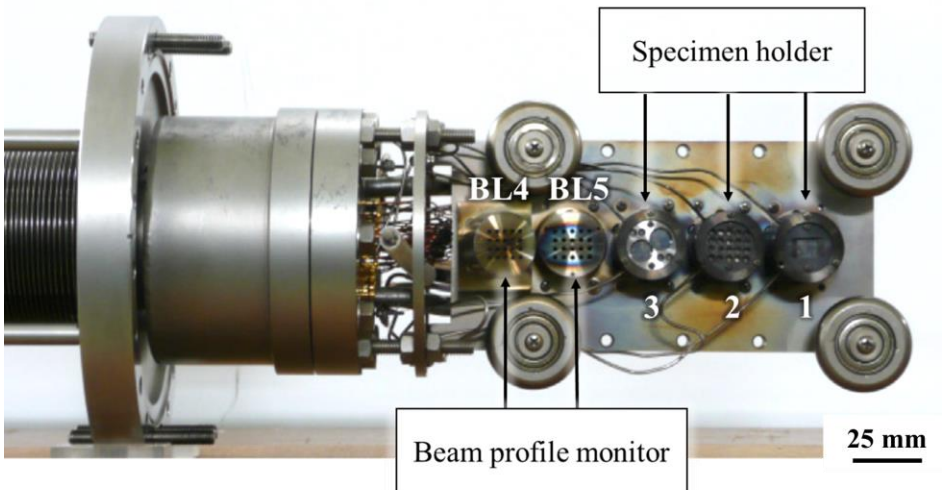
HIT: High Fluence Irradiation Facility of The Univ. of Tokyo

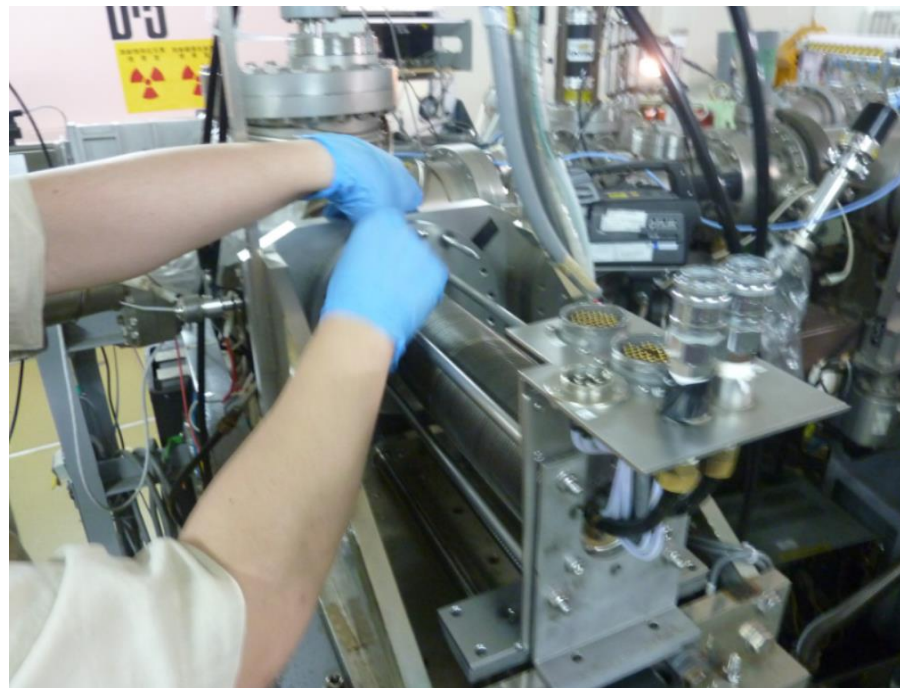
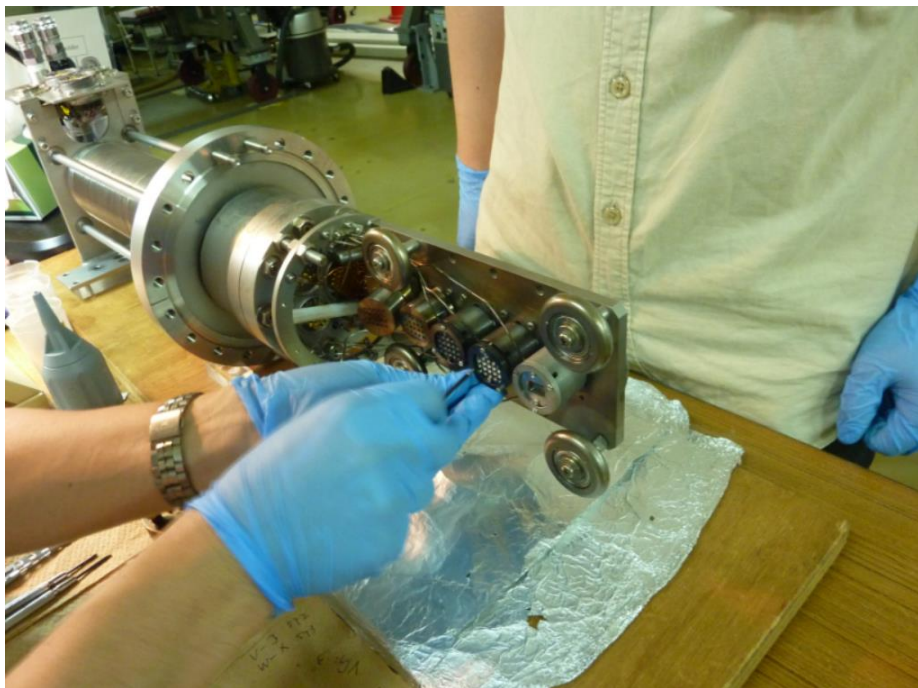
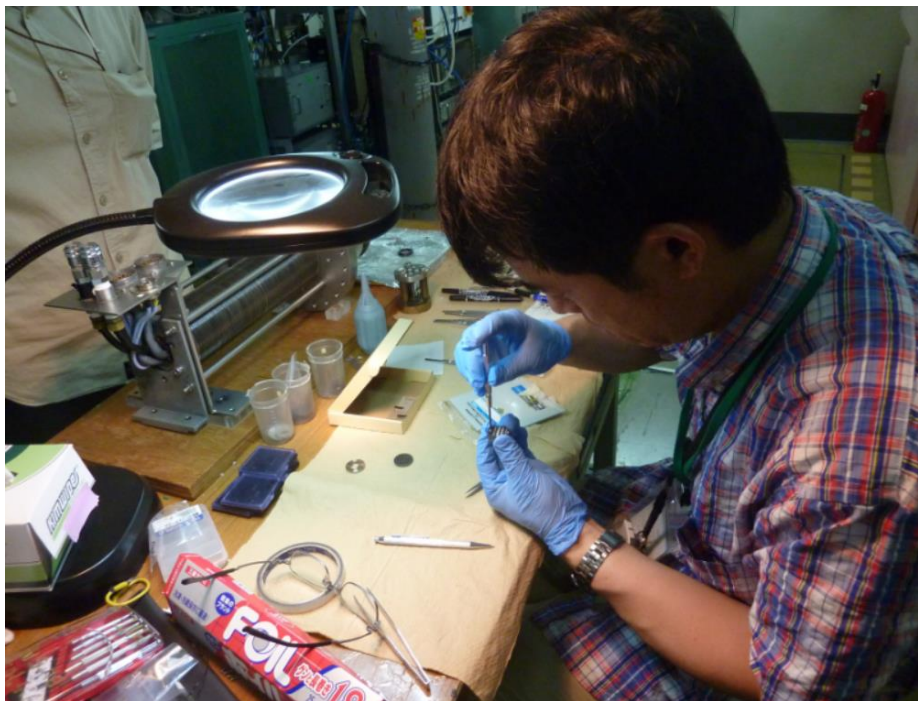


Depth from Surface (μm)
Current : ~8nA
Dose rate : 9.5x10⁻⁴ dpa/s



Depth from Surface (μm)
Current : ~8nA
Dose rate : 3.3x10⁻³ dpa/s





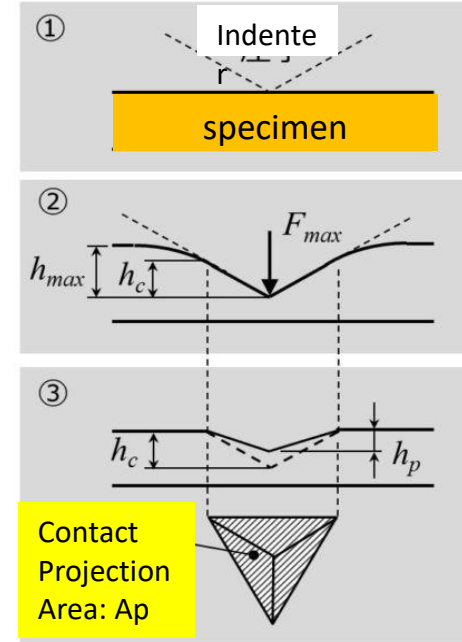
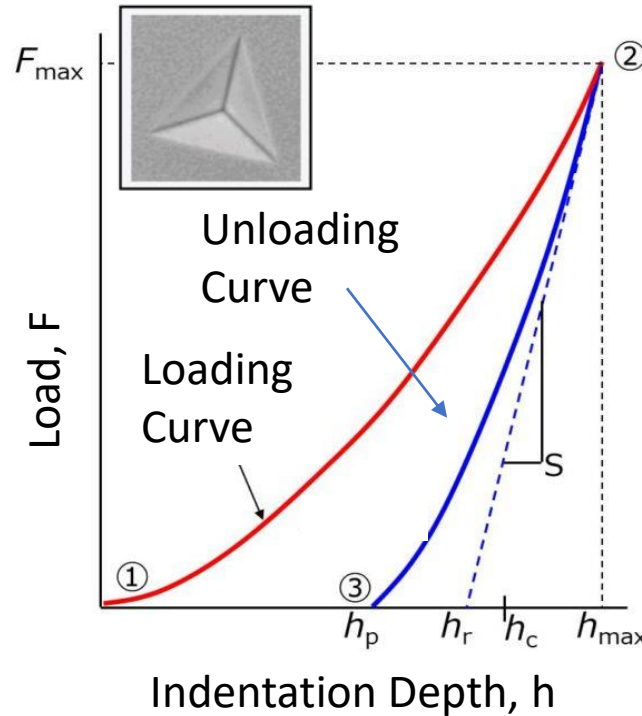
HIT Ion Irradiation Experiment's Matrix in August 2019 (3 days)

Specimen	1 dpa	5 dpa	10 dpa
Ti-15V-3Cr-3Al-3Sn Solution Treatment, not Aged (ST)	○	○	○
Ti-15V-3Cr-3Al-3Sn Solution Treatment and Aged (STA)	○	○	○
Ti-6Al-4V Gr23 ELI Annealed (A)	○	○	○
Ti-6Al-4V Gr23 ELI STA (WQ -> Aging)	○	○	○
W-TFGR	-	-	○
Pure-W	-	-	○

W-TFGR: “Toughened Fine-Grained Recrystallized” Highly-ductile Tungsten → S. Makimura

Nano Indentation Test

EIT Analysis (EIT: Elastic modulus of Indentation Testing)



$$h_c = h_{max} - \epsilon(h_{max} - h_r)$$

$$H_{IT} = \frac{F_{max}}{A_p(h_c)}$$

E_r : Elastic modulus of composite of Indenter and specimen

$$E_r = \frac{S\sqrt{\pi}}{2\sqrt{A_p(h_c)}}$$

- The indentation modulus **EIT** of the specimen material is related to the reduced elastic modulus E_r and the indenter modulus E_i :

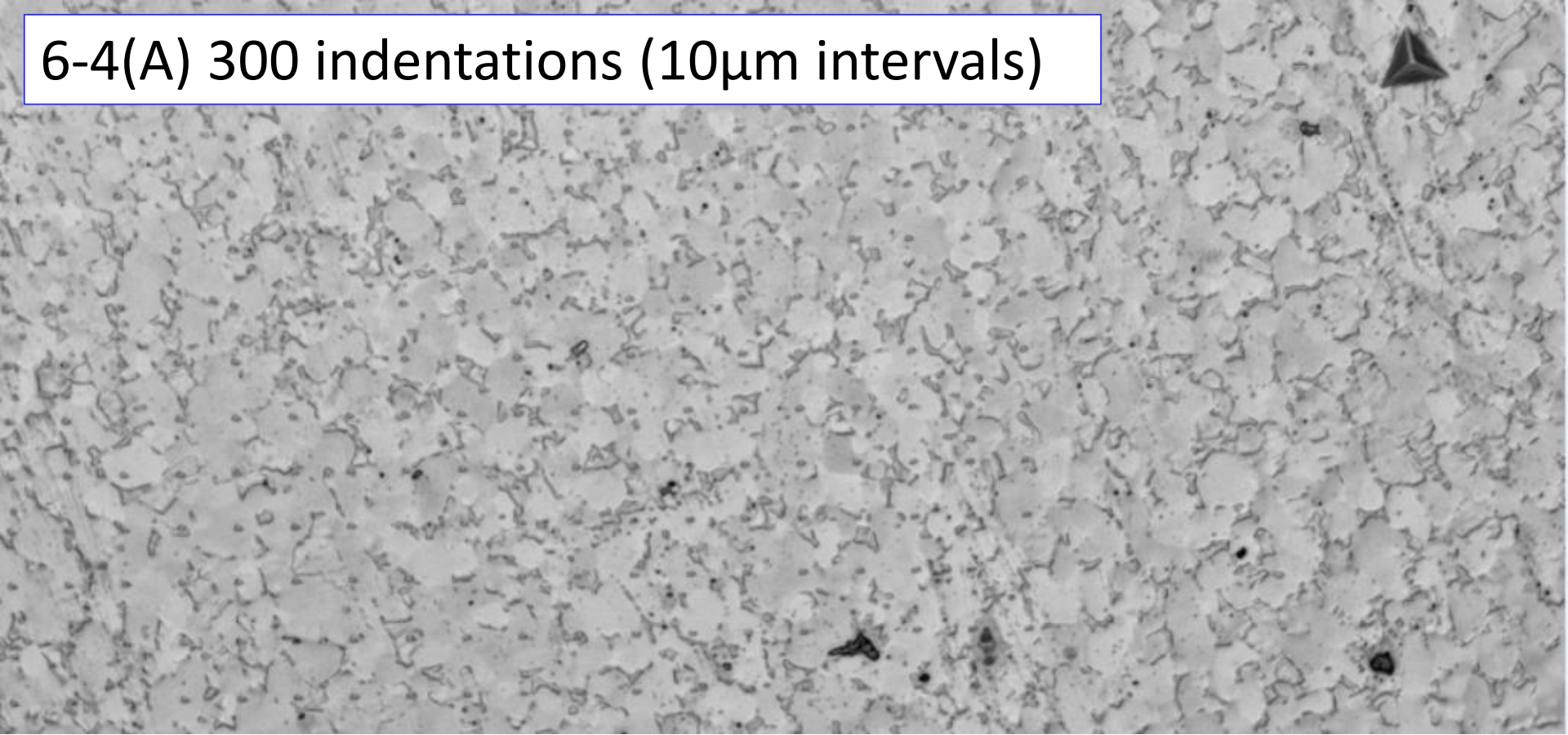
$$E_{IT} = \frac{1 - (\nu_s)^2}{\frac{1}{E_r} - \frac{1 - (\nu_i)^2}{E_i}}$$

E_i, ν_i : Elastic modulus, Poisson's ratio of Indenter
 ν_s : Poisson's ratio of specimen,

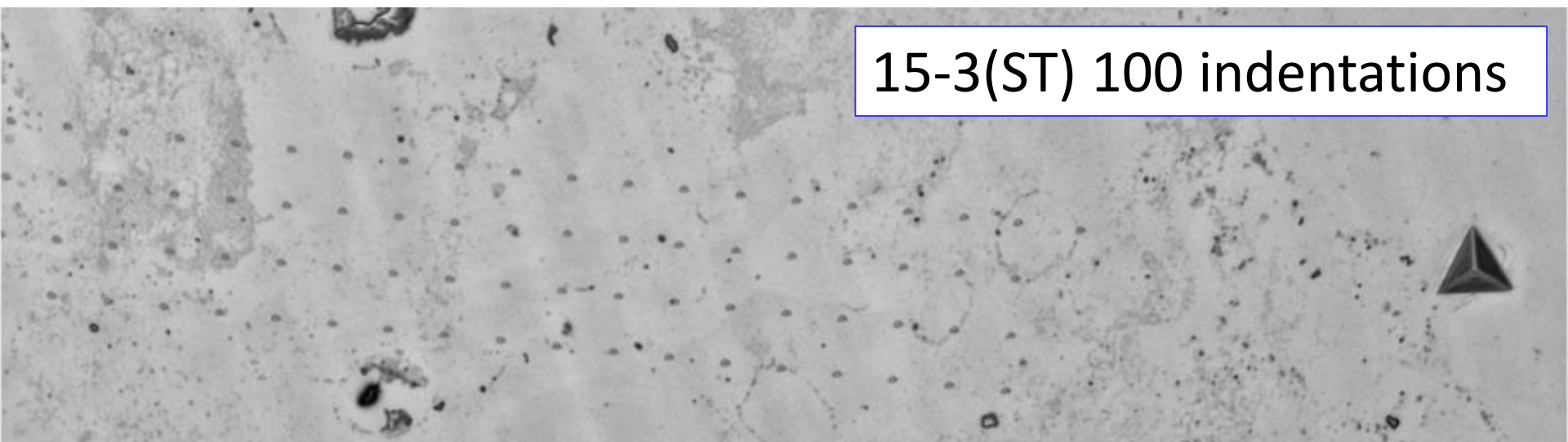
Shimadzu- DUH-211S type, set in the University of Tokyo

Indenter : Berkovich(115°)
Indentation depth :150nm (fix)

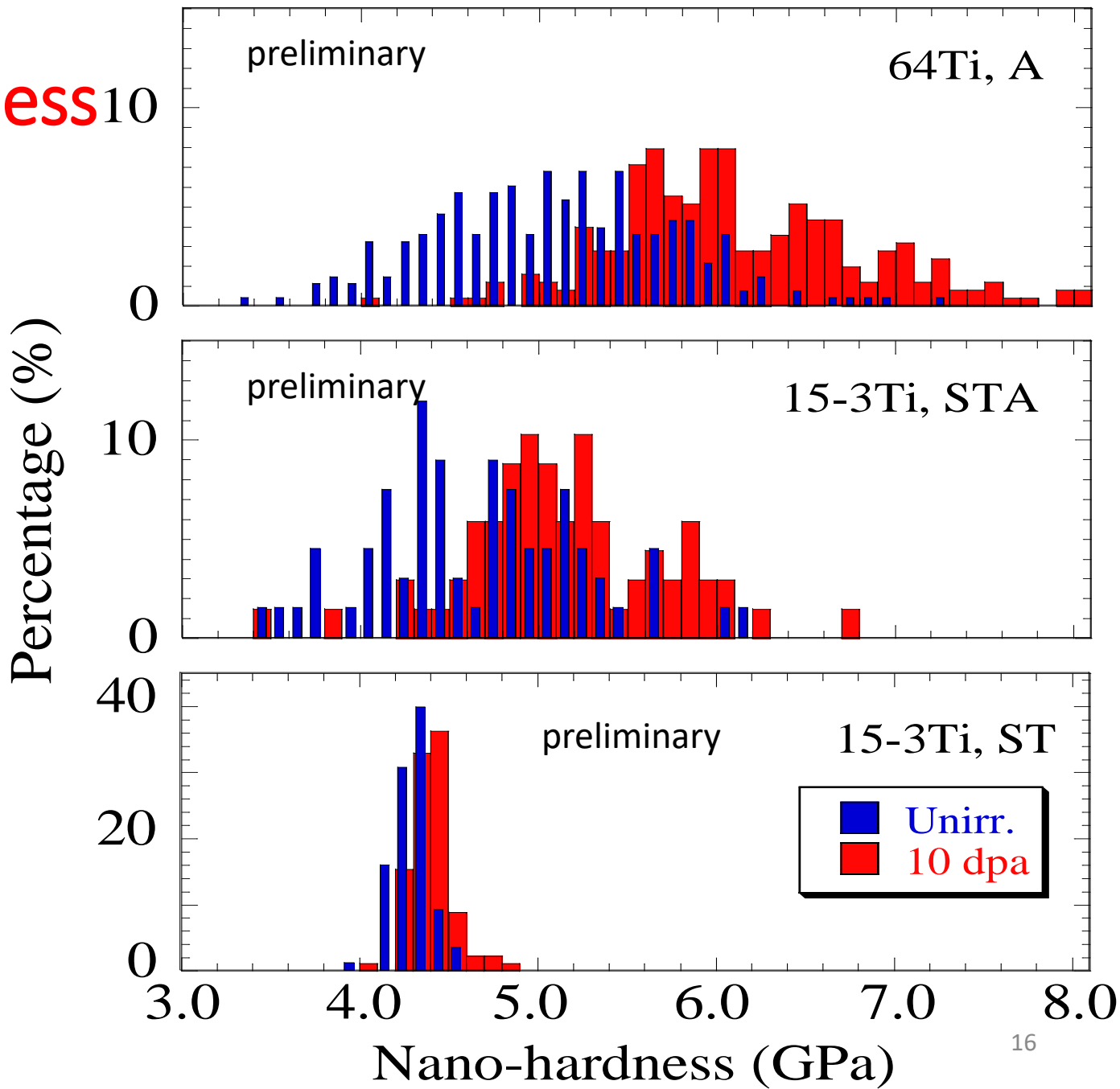
6-4(A) 300 indentations (10 μ m intervals)



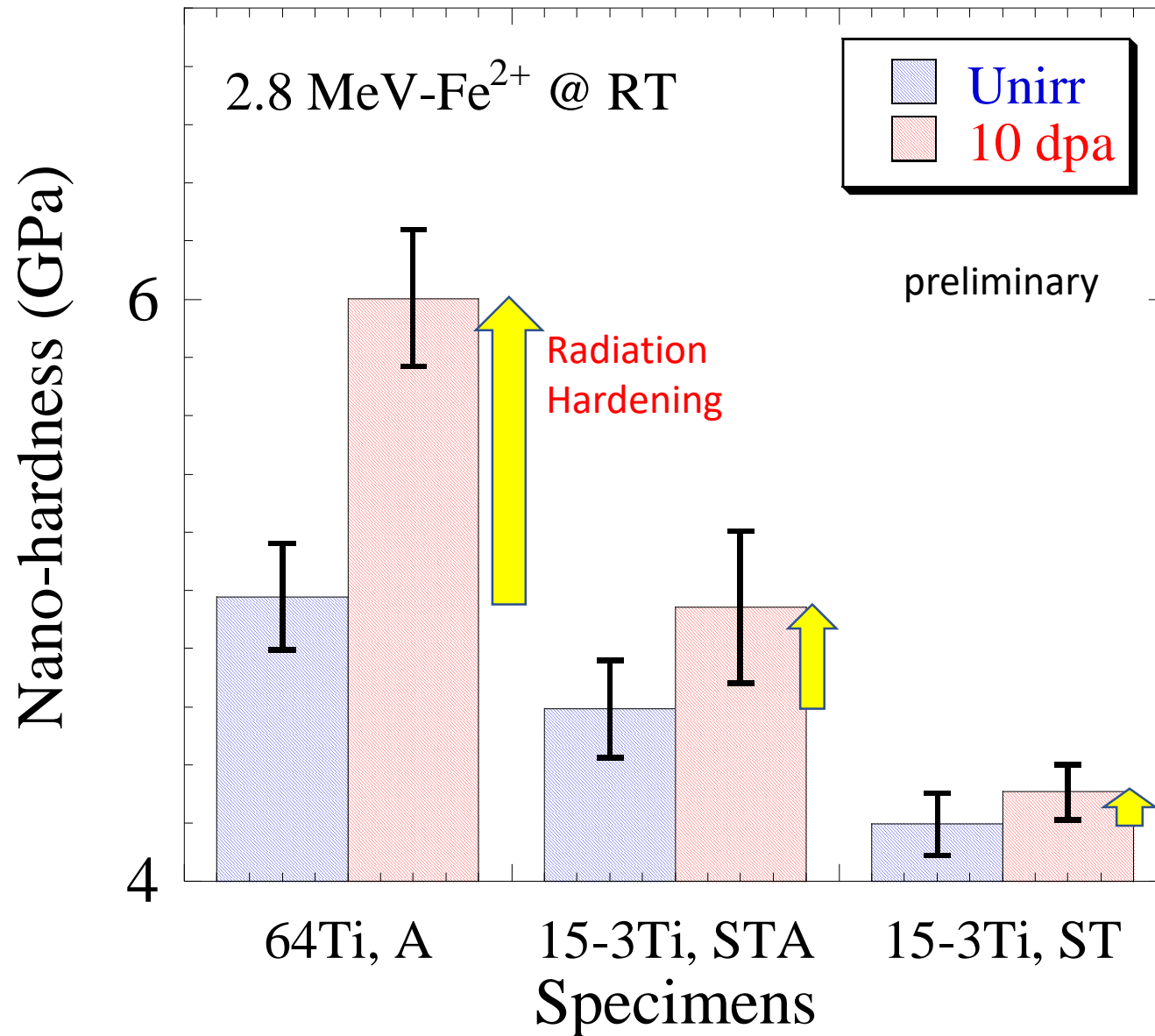
15-3(ST) 100 indentations



Result: Nano-hardness Distribution



Result: Ti Alloys – Averaged Nano-Hardness



Crystal orientation effect

Scripta Materialia 162 (2019) 209–213

Contents lists available at ScienceDirect

Scripta Materialia

journal homepage: www.elsevier.com/locate/scriptamat



Surface orientation dependence of irradiation-induced hardening in a polycrystalline zirconium alloy

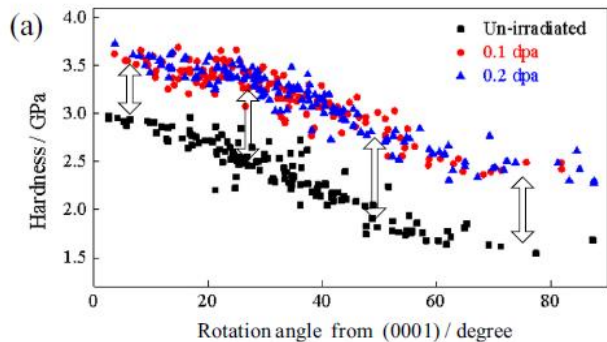
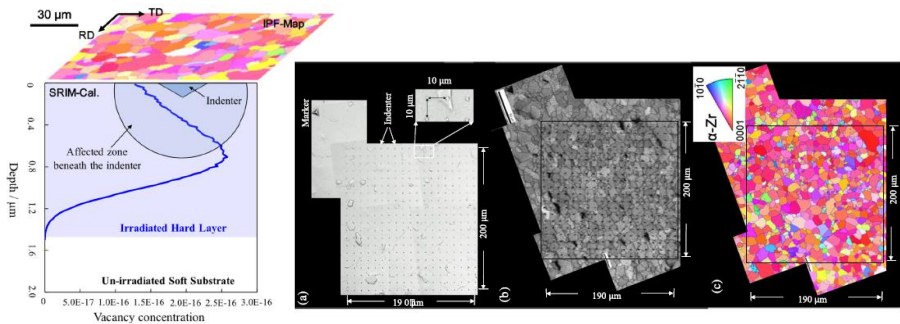
H.L. Yang ^{a,*}, S. Kano ^a, J. McGrady ^a, J.J. Shen ^{a,1}, Y. Matsukawa ^b, D.Y. Chen ^c, K. Murakami ^d, H. Abe ^{a,*}

^a Nuclear Professional School, School of Engineering, The University of Tokyo, 2-22 Shirokata Shirane, Tokai, Ibaraki 319-1188, Japan

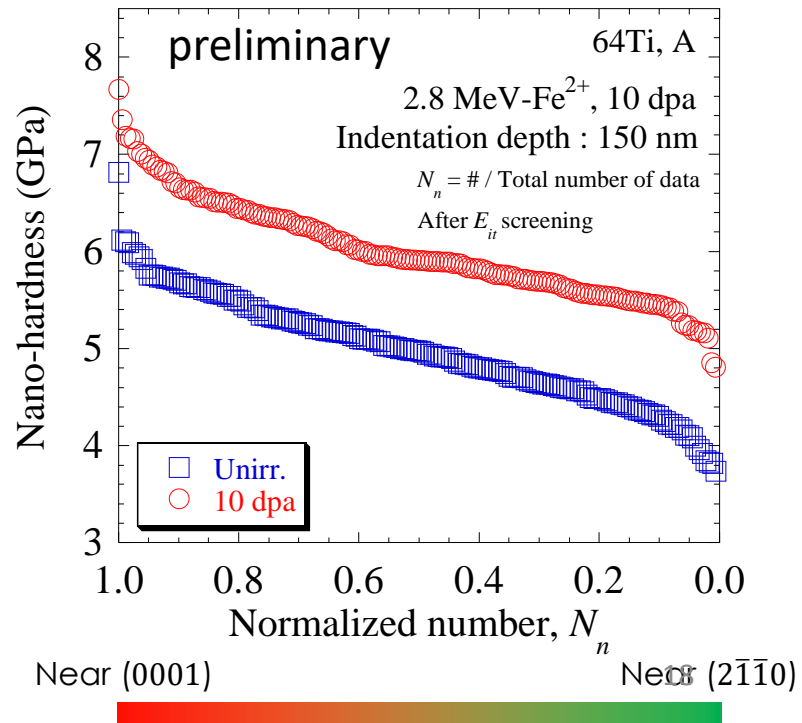
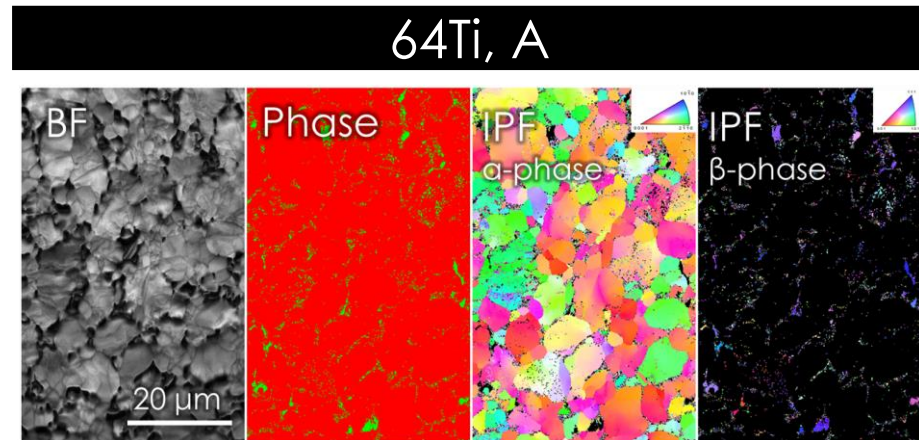
^b Graduate School of Engineering, Tohoku University, 2-1-1 Katahira, Sendai 980-8577, Japan

^c Department of Nuclear Engineering and Management, School of Engineering, The University of Tokyo, 7-3-1, Tokyo, Hongo, Bunkyo, 113-8656, Japan

^d Institute of Giga-Technology, Nagasaki University of Technology, 1603-1 Kamiamatsuka, Nagasaki 840-2188, Japan

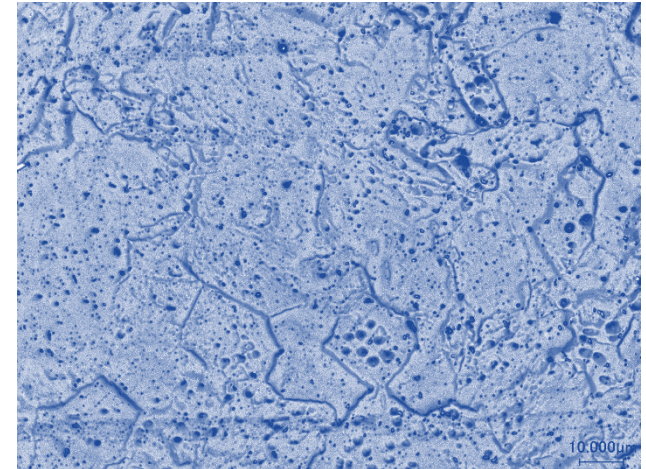


- Nano-hardness in hcp material is strong affected by the crystal texture.

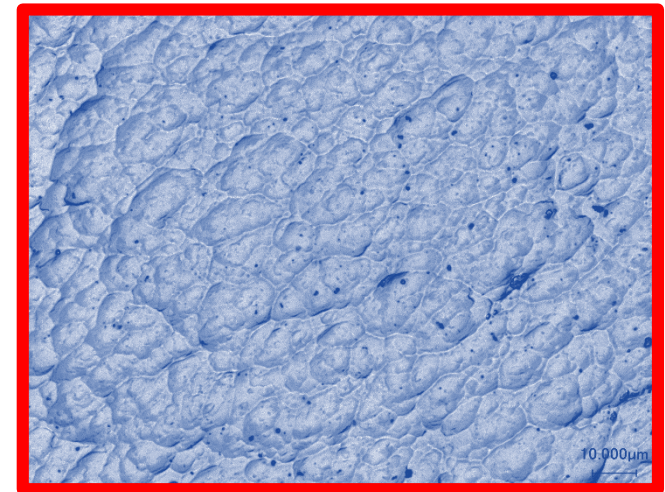


TFGR-W irradiation test

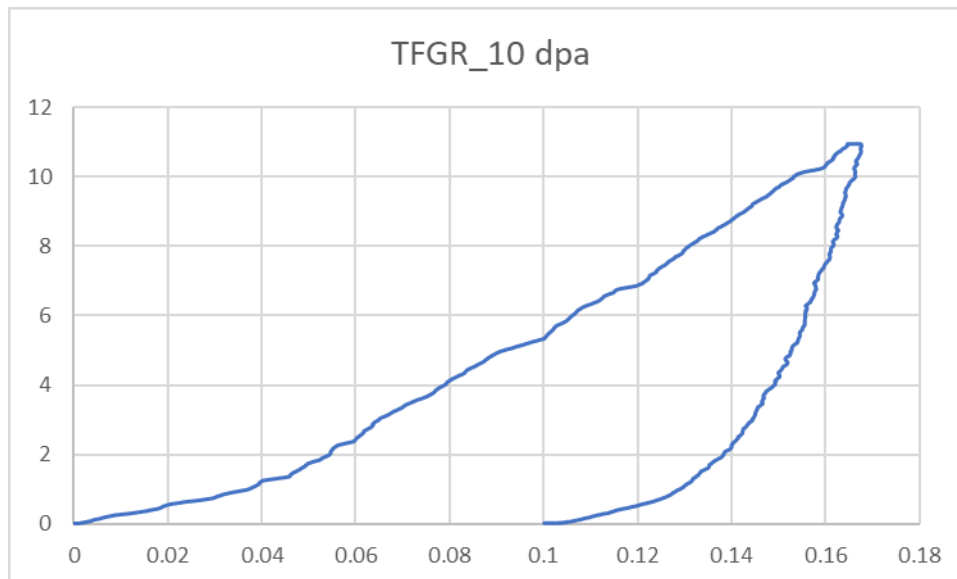
- Electro-polishing process could not be optimized on time.
- Many pivots were observed on the specimen surface
- Load-strain curve is not smooth, probably because indenter slides on surface
- Improvements on electro-polish process are on-going
- Results to be obtained at next irradiation scheduled on March 2020.



Pure tungsten: x1000



TFGR tungsten: x1000



Summary

1. High-DPA ion beam irradiation experiment has been conducted at the HIT facility on Titanium alloys and Tungsten alloys: $\alpha+\beta$ 64-Ti, metastable- β 15-3-Ti, pure W and TFGR-W were irradiated to 1, 5, and 10 dpa (W: 10 dpa only) by Fe^{2+} at ambient temperature.
2. Preliminary results of nano-hardness measurements for 64-Ti(A), 15-3-Ti (ST), and 15-3-Ti(STA) irradiated to 10 dpa were reported.
3. The radiation hardening of 15-3-Ti alloys were significantly smaller than that for 64-Ti alloy. Especially, hardness of 15-3-Ti(ST) stays the same within the statistical error after the irradiation to 10 dpa.
4. This may suggest that the metastable- β Ti alloys exhibit high defect sink strength, because of their nature to employ “precipitation strengthening”, nano-scale precipitates act as defect sink sites.
5. Next irradiation at the HIT facility will be conducted in March 2020 on aerospace grade metastable β Ti alloy(s), which has better creep property to higher temperature than 15-3 Ti (200°C).
6. The TFGR-W will also be examined with optimized polishing.

Thank you for your attention.